

Review of Spent Nuclear Fuel Dry Storage Demonstration Programs in US

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1. Introduction

US have carried out several notable demonstration programs for the initiation and license extension of dry storage of spent nuclear fuel (SNF). At the very early stage of dry storage, the demonstration programs were focused on the proof of the safety of dry storage systems and a demonstration project named dry cask storage characterization project (DCSCP) was performed for the license extension of low burn-up fuel dry storage. Currently, a demonstration program for the license extension of high burn-up fuel dry storage is under way and it is expected to continue at least 10 years from now. Korea has not begun the dry storage of PWR fuel yet and those programs of US can be a good reference and lessons to safely begin and operate dry storage in Korea. In this paper, the past and current of the demonstration programs of US are analyzed and several recommendations are provided on the demonstration programs for dry storage of spent nuclear fuel in Korea.

2. Regulations on SNF Integrity

2.1 Storage regulations

The regulatory requirements on the integrity of spent nuclear fuel are stipulated in 10 CFR 72, NRC Interim Staff Guidance (ISG) and NUREG documents. 10 CFR 72.122(h) states that (1) The spent fuel cladding must be protected during storage against degradation that leads to gross ruptures or the fuel must be otherwise confined such that degradation of the fuel during storage will not pose operational safety problems with respect to its removal from storage, and (5) The high-level radioactive waste and reactor-related GTCC waste must be packaged in a manner that allows handling and retrievability without the release of radioactive materials to the environment or radiation exposures in excess of part 20 limits. The definition of the term “retrievability” is defined in ISG-2 which was

recently revised in 2016 to accommodate the retrieval of spent fuel in canister and storage cask. The requirements on the retrievability do not apply to the accident conditions.

2.2 Transportation regulations

In the transportation regulation, 10 CFR 71, the requirements on the integrity of spent nuclear fuel is not explicitly stipulated. In the regulations for fissile material transportation, 10 CFR 71.55, it states that (1) The contents would be subcritical; (2) The geometric form of the package contents would not be substantially altered in normal conditions of transportation. As the degradation of fuel cladding and other structural parts of fuel assembly may lead to the gross rupture of fuel assembly in the presence of transportation loading such as vibration, the requirements on transportability shares some aspects with the requirements on retrievability.

3. SNF Dry Storage Demonstration Program

3.1 Demonstration at early stages

The purpose of the demonstration programs at early 1980's was the verification of the performance and safety of the dry storage systems and set-up of the dry storage procedure at reactor site. This early stage demonstration was initiated by the Nuclear Waste Policy Act of 1982. It includes the demonstration with the Sealed Storage cask (Fig. 1) at Nevada Test Site (NTS) and the demonstration performed at Surry nuclear power plant with CASTOR V/21 cask, demonstration at H.B. Robinson power plant with NUHOMS system and the demonstration at INEEL (currently INL) with several types of casks (Fig. 3). Through these demonstration programs, the DOE staffs successfully validated and verified their computational model for shielding and computational fluid dynamic codes such as COBRA-SFS, HYDRA.

