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The Design Optimization of Preventive Measure Against APR1400 Steam Generator Tube Fretting Wear

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Key Words: APR1400 Steam Generator(1400), Inconel-690(-690), F.I.V(), Egg-crate Flow Distribution Plate()

Abstract

Inconel-600 alloy has been used as steam generator tube material for current pressurized water reactors (PWRs). The long-term operation of steam generators showed that the use of this material induced localized corrosion damages and increased tube wear of steam generator. To protect these problems, steam generator tube material is being changed to Inconel-690 alloy. Based on the current trend, we have chosen Inconel 690 as the Advanced Power Reactor 1400 (APR1400) steam generator(SG) tube material and performed the design optimization of preventive measure against tube fretting wear for the APR1400 steam generator. In this paper, we examined the technical consideration in this modification : the selection of material, wear characteristics, effect of the Egg-crate Flow Distribution Plate installation, and effect analysis of vertical strip installation.

1 2

EFDP : Egg-crate Flow Distribution Plate
F.I.V : Flow-induced Vibration
UBS : Upper Bundle Support
RMS : Root Mean Squire

1. I-600 가

2. I-600 690

+ ()
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2. 1400

APR1400 (KSNP)

U-bend

U-bend
Full egg- crate
(Egg-crate

Flow Distribution Plate : EFDP)

가

U-bend

Vertical Strip Horizontal Strip 가
가

1976 ~ 1979

Denting

1979

(ODSCC/IGA)
(PWSCC)

1

1983

가

(Central Cavity)

[1]

3.

3.1

3.1.1

(Denting)

(Stress Corrosion Cracking)

(Fretting)

(Pitting)

(Thinning)

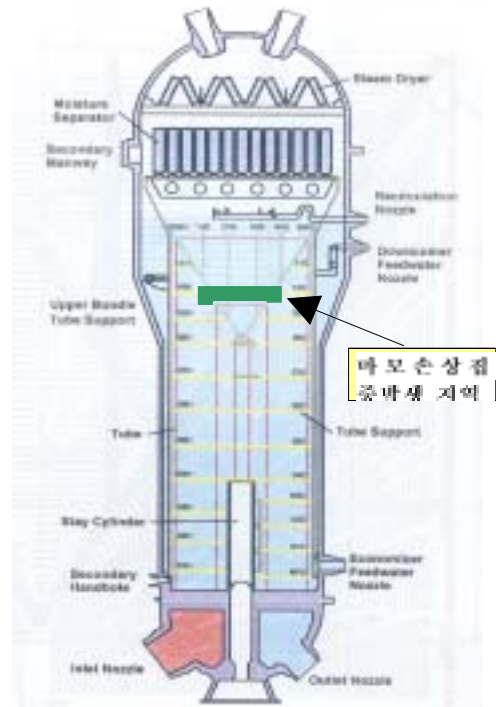


Fig. 1 Region of fretting wear in S/G tube

1

3.3

1

2

3.2

1

2

가

1976

Thinning 가

Table1. Chemical composition(%) of I-600 and I-690

계 질	Composition, Wt %							
	Fe	Ni	Cr	C	Mn	Si	Al	Ti
I-600	8	72	15	.03	.2	.1	.2	.25
I-690	10	58	30	.02	.2	.1	.2	.25

3.3.1 가 (Fretting Wear) 가 가

3.3.2 (Impact-Fretting Wear) 가 가

4.1.2

I-600 I-690 (50 500 가)

10 ~ 30N I-600/405

I-690/409 [2,3,4] 가 가

690

4. S/G

4.1

I-690

4.1.1

가

I-600

Ni-Cr-Fe

I-690

I-600

I-690

I-690

I-600

Ni

Cr

2

가

[1]

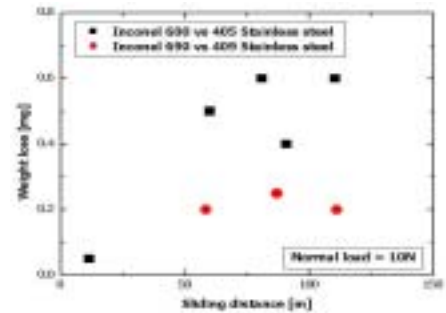


Fig. 2 Comparison of wear quantity (Vertical load : 10N)

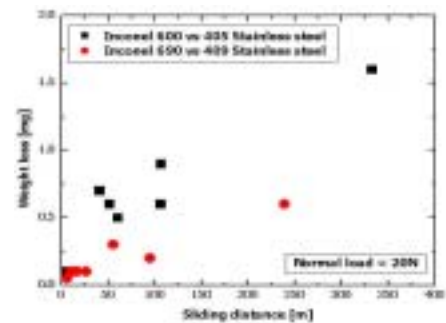


Fig. 3 Comparison of wear quantity (Vertical load : 20N)

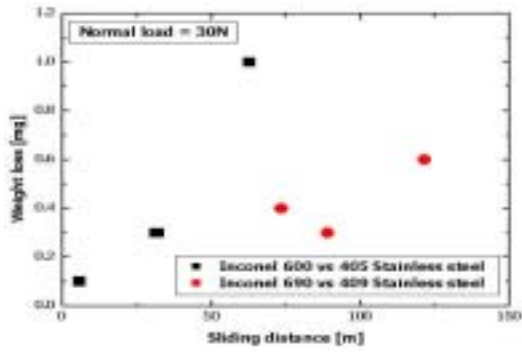


Fig. 4 Comparison of wear quantity (Vertical load : 30N)



Fig. 6 Observation of worn surface in I-600

4.1.3

가 가

I-690

, I-690/409

5

I-690/409

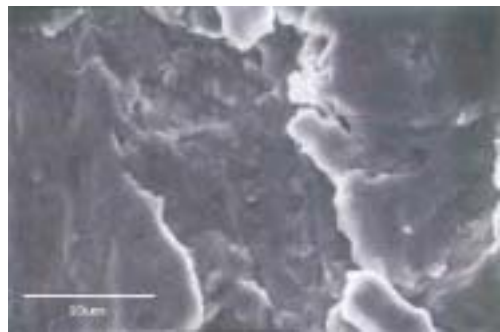


Fig. 7 Observation of worn surface in I-690

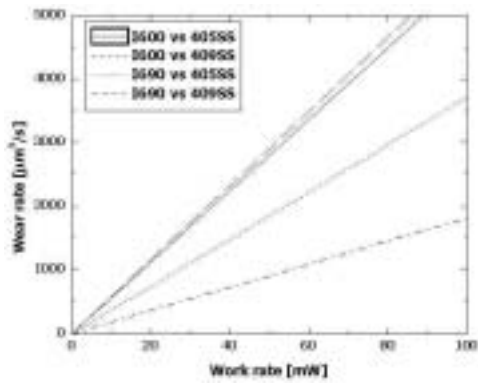


Fig. 5 Wear coefficient comparison of I-600 & I-690

4.2

(Tube Span

)

가

1400

가

4.1.4

(SEM)

. I-600

가

, I-690

가

가

I-690

. [6,7]

4.2.1

가

4.2.2

(EFDP)

(EFDP)

. [8]

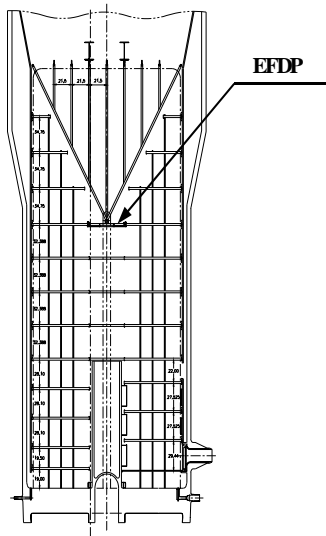


Fig. 8 EFDP of the Steam Generator

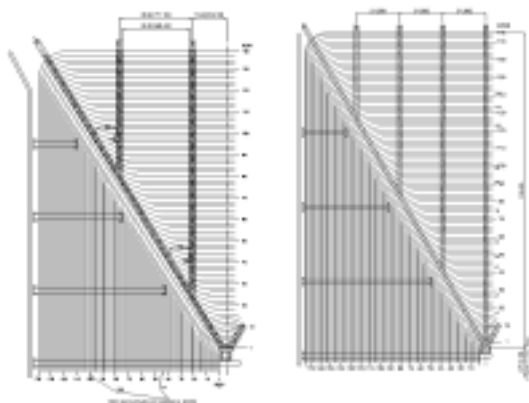
4.2.3 UBS(Upper Bundle Support)

UBS(Upper Bundle Support)

Grid Horizontal Grid [9] 가

Vertical

Vertical



(a) 개선전

(b) 개선후

Fig. 9 Upper tube bundle supports before and after design improvement

4.3

4.3.1 가

가

가

가

가

가

2

가

2

가
2
가
1400
가
10%
가
2
가
2

Table2. Average velocity of Egg.

변수		개선전	개선후	변화 (%)
2	(10 ⁶ lb/hr)	25.10	35.35	40.9
유동 면적 (ft ²)		122.0	186.3	52.7
평균 속도 (ft/sec)	Full Eggcrate #5	3.34	2.81	-15.88
	Partial Eggcrate #1	3.65	3.20	-12.20
	Partial Eggcrate #2	4.04	3.68	-8.90
	Partial Eggcrate #3	4.17	3.81	-8.54

4.3.2

(EFDP)

EFDP

[3]

, (IZ=30

) 가

Dynamic Pressure

42%

가

Table3. Thermal-hydraulic analysis results

구 분	개선전	개선후	증감율 (%)
Density, lb/ft ³	21.44	18.70	- 12.8
Velocity, ft/sec	4.50	3.67	- 18.4
Mass Flux, lbm/ft ² -sec	96.59	68.54	- 29.0
Dynamic Pressure, psi	0.0469	0.0272	- 42.0

4.3.3 (UBS) (EFDP) Stability Ratio RMS
 가 가 가
 [9] Vertical Support Vertical Strip
 F.I.V 가 S/G I-600 I-690
 4] 가 [가
 78%, 95%가

Table4. Summary of design improvement effect

Design	Stability Ratio	최대진동 변위 (mils)	비고
표준설계	0.730	26	SR !고값 : 1.0 하
개선후	0.162	1.3	최대진동 변위권고값 : 10mils 하
개선효과	77.8%	95.0%	

5.

(CE) I-600

I-690/409
 (Flow-induced Vibration) 가
 1400 가
 가 가
 78%, 95%

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